

EUROFUSION WPJET1-PR(15)20

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# Comparative Gyrokinetic Analysis of JET Baseline H-mode Core Plasmas with Carbon Wall and ITER-Like Wall

Preprint of Paper to be submitted for publication in Plasma Physics and Controlled Fusion



This work has been carried out within the framework of the EUROfusion Consortium and has received funding from the Euratom research and training programme 2014-2018 under grant agreement No 633053. The views and opinions expressed herein do not necessarily reflect those of the European Commission. "This document is intended for publication in the open literature. It is made available on the clear understanding that it may not be further circulated and extracts or references may not be published prior to publication of the original when applicable, or without the consent of the Publications Officer, EUROfusion Programme Management Unit, Culham Science Centre, Abingdon, Oxon, OX14 3DB, UK or e-mail Publications.Officer@euro-fusion.org".

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### <sup>1</sup> Comparative gyrokinetic analysis of JET baseline

## <sup>2</sup> H-mode core plasmas with carbon wall and

### <sup>3</sup> ITER-like wall

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#### 12 Abstract

Following the change of plasma facing components at JET from a carbon wall (CW) to a 13 metal ITER-like wall (ILW) a deterioration of global confinement has been observed for 14 H-mode baseline experiments. The deterioration has been correlated with a degradation 15 of pedestal confinement with lower electron temperatures at the top of the edge barrier 16 region. In order to investigate the change in core confinement, heat transport due to Ion 17 Temperature Gradient (ITG)/Trapped Electron Mode (TEM) turbulence is investigated 18 using the gyrokinetic code GENE. Two pairs of CW and ILW discharges that are 19 matched according to several global parameters are simulated at mid radius. The 20 simulations included effects of collisions, finite  $\beta$ , realistic geometries, and impurities. 21 A sensitivity study is performed with respect to the key dimensionless parameters in 22 the matched pairs. The combined effect of the relative change in these parameters is that 23 the ITG mode is destabilized in the ILW discharges compared to the CW discharges. 24 This is also reflected in nonlinear simulations where the ILW discharges show higher 25 normalized ion and electron heat fluxes and larger stiffness. The ion energy confinement 26 time within  $\rho = 0.5$  is found to be comparable while the electron confinement time is 27 shorter for the ILW discharges. The core confinement in the ILW discharges is expected 28 to improve if the edge pedestal is recovered since that would favourably change the key 29 plasma parameters that now serve to destabilize them. 30

‡ See the Appendix of F. Romanelli et al., Proceedings of the 25th IAEA Fusion Energy Conference 2014, Saint Petersburg, Russia

#### 31 1. Introduction

Initial studies indicate that the interaction between the hot fusion plasma and the 32 surrounding wall in magnetic fusion confinement devices can influence key plasma 33 performance parameters like the energy confinement time. With the new ITER-like 34 wall (ILW) at JET [1], the carbon wall (CW) has been replaced by a metal beryllium 35 first wall and a tungsten divertor. To study the effect of the new ILW on confinement, 36 a database has been created comprising a set of JET discharges with ILW and matched 37 C-wall shots using the same criteria as in [2]. The database contains both baseline 38 H-mode and hybrid scenarios, at high and low delta. The ILW experimental program 39 has already produced many interesting results which are not well understood [2, 3]. In 40 particular a deterioration in global confinement has been observed at JET in baseline 41 H-mode experiments following the change from a from a CW to an ILW [4]. One 42 cause of the deterioration is the high deuterium gas puffing rate necessary in ILW 43 discharges in order to mitigate W accumulation. For low triangularity plasmas, this 44 degradation of confinement with fuelling level was also previously observed for CW 45 The deterioration has been correlated by a degradation of pedestal discharges [5]. 46 confinement with lower electron temperatures at the top of the edge barrier region. 47 This leads to lower electron temperature in the core, thereby changing the NBI heat 48 deposition profiles in the core. As a result, the core energy confinement time has been 49 influenced with lower electron energy confinement time and similar ion confinement 50 time in the ILW case [4]. In the present work, gyrokinetic modelling of similar CW and 51 ILW discharges is carried out in order to assess the differences seen in core confinement. 52 The discharges have ion temperature data available and have been selected in order 53 to match the average value of global controllable parameters within a reference time 54 window during the flat top. Parameters are taken from interpretative TRANSP [6, 7] 55 Transport due to Ion Temperature Gradient (ITG) /Trapped electron simulations. 56 mode (TEM) [8, 9, 10, 11, 12, 13, 14] turbulence is calculated using the gyrokinetic 57 code GENE [15]. Both linear and nonlinear simulations are performed in a flux tube 58 domain. The gyrokinetic simulations include finite  $\beta$  effects, collisions, impurities, and 59 rotational effects in realistic geometry. Linear sensitivity scans for the paired discharges 60 are performed for plasma  $\beta$ , collisionality, safety factor, magnetic shear, impurity content 61 and electron and ion temperature gradient. The differences in the energy flux and energy 62 confinement times are investigated using nonlinear GENE simulations. 63

The remainder of the paper is organized as follows: In Sec. 2 the gyrokinetic model and the input parameters used are introduced. In Sec. 3 the linear sensitivity results are presented, followed by the nonlinear results in Sec. 4. Finally, in Sec. 5 follow the concluding remarks.

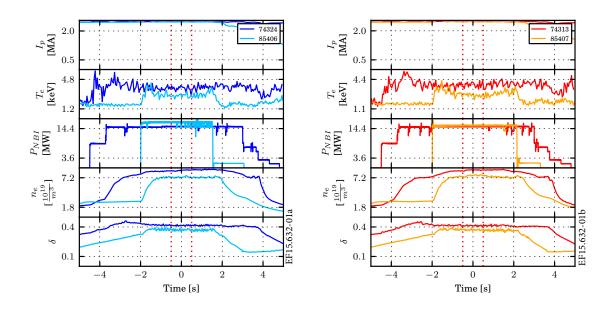
Shot number	$B(\mathbf{T})$	$T_e \; (\mathrm{keV})$	$T_i \; (\mathrm{keV})$	$n_e (10^{19}/{ m m}^3)$	$\Omega_{tor} (\mathrm{krads}^{-1})$
74313	2.62	2.31	2.11	9.04	32
85407	2.68	1.70	1.71	8.19	26
74324	2.64	2.35	2.08	8.72	31
85406	2.68	1.78	1.75	7.56	31

Table 1: Discharge dimensional parameters of the four discharges.

#### <sup>68</sup> 2. GENE simulations setup and discharge parameters

GENE solves the nonlinear gyrokinetic Vlasov equations together with Maxwell's 69 equations in order to find the distribution functions of the species,  $f(\mathbf{R}, v_{\parallel}, \mu, t)$ , 70 the electrostatic potential,  $\phi(\mathbf{x}, t)$  and the parallel components of the magnetic vector 71 potential and magnetic field,  $A_{\parallel}(\mathbf{x},t)$  and  $B_{\parallel}(\mathbf{x},t)$ . The coordinate system is aligned 72 to the background magnetic field with x as the radial coordinate, y as the binormal 73 coordinate, and z as the parallel coordinate. Collisions are modelled using a linearised 74 Landau-Boltzmann collision operator [16]. Magnetic fluctuations were included in all 75 simulations. The pressure gradient, as used in the calculation of the curvature and  $\nabla B$ 76 drift, is set to be consistent with the density and temperature gradients and the plasma 77  $\beta$ . In this work, the Miller geometry model [17] is used. The Miller geometry model 78 allows the magnetic geometry to be completely described by nine parameters. These 79 parameters were extracted from numerical geometries reconstructed by the EFIT code 80 [18]. For the linear simulations both an initial value solver and an eigenvalue solver that 81 can find subdominant modes are used. 82

Two ITER-like wall discharges and two C-wall discharges with global parameters 83 matched as closely as possible are analysed. The matched global parameters 84 are the plasma current, the toroidal magnetic field, applied NBI power, average 85 electron density, safety factor, and triangularity. The discharges are baseline H-mode 86 with ion temperature and rotation measurements available through charge exchange 87 spectroscopy. Discharge parameters are taken from TRANSP runs [7, 6] performed 88 with electron density and temperature profiles from high resolution Thomson scattering 89 measurements. One impurity species is included in the simulation, carbon for the carbon 90 wall discharges and beryllium for the ITER-like wall discharges. The impurity density 91 was calculated from  $Z_{eff}$ , which is assumed to be constant over the whole radius [4]. The 92 four discharges are analysed at  $\rho = 0.5$  where  $\rho$  is the normalized toroidal flux coordinate. 93 The baseline H-mode discharges are pair wise 74313 (CW), 85407 (ILW), 74324 (CW) 94 and 85406 (ILW). In Figure 1 the time evolution of the discharges is shown. The relevant 95 discharge parameters are shown in Table 1 (dimensional) and 2 (dimensionless). Radial 96 temperature, density and rotational speed profiles are shown in Figure 2. The data is 97 averaged over a one second time window and further smoothed in the radial direction. 98



(a) CW discharge 74324 and ILW discharge 85407(b) CW discharge 74313 and ILW discharge 85406

Figure 1: Time evolution of the two pairs of matched discharges. Time point of analysis indicated with 0.

Shot	$\hat{s}$	q	$T_i/T_e$	$R/L_{T_i}$	$R/L_{T_e}$	$\beta$ (%)	$\nu_c(10^{-3})$	$Z_{eff}$	$\gamma_{\mathbf{E}\times\mathbf{B}}$	δ
number										
74313	0.56	1.42	0.92	6.56	6.19	1.2	1.8	1.58	0.056	0.097
85407	0.66	1.32	1.00	5.96	8.28	0.78	3.0	1.05	0.10	0.081
74324	0.55	1.44	0.89	4.92	5.96	1.19	1.7	1.56	0.040	0.097
85406	0.64	1.34	0.98	6.78	8.38	0.75	2.5	1.05	0.22	0.083

Table 2: Discharge dimensionless parameters at  $\rho = 0.5$ . Collision frequency calculated as  $\nu_c = \pi \ln \Lambda e^4 n_e R / (2^{3/2} T_e^2)$ .

#### 99 3. Linear results

The computational parameters used in the linear simulations are a resolution of  $32 \times 24$  in 100 the parallel and normal direction with 64 grid points in the parallel velocity direction and 101 16 magnetic moments. An initial value solver is typically used, in the cases where sub 102 dominant modes are presented an eigenvalue solver is used. The linear ITG/TE mode 103 stability of the two matched pairs is investigated at mid radius. Due to the experimental 104 uncertainty in the value of  $R/L_{T_i}$ , the linear results are displayed in a scan over  $R/L_{T_i}$ . 105 Figure 3a shows the growth rates and Fig. 3b the corresponding eigenfrequencies at 106  $k_u \rho_s = 0.3$ . As observed, the turbulence is ITG dominated for  $R/L_{T_i} > 4$  ( $\omega_r > 0$ ) for 107 the ILW discharges and TEM dominated for lower  $R/L_{T_i}$  while for the CW discharges 108

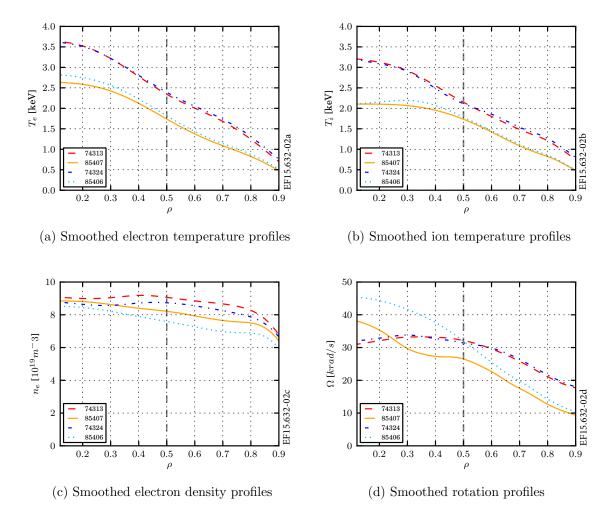


Figure 2: Density and temperature profiles from time averaged and smoothed TRANSP data. Values are averaged between 11.5 s and 12.5 s for the C-wall discharges and 19.5 s and 20.5 s for the ITER-like wall discharges.

the TE mode is not excited. We have verified that the results are similar for other values 109 of  $k_u \rho_s$  around the maximum growth rate which occurs at around  $k_u \rho_s = 0.3$ . The ITG 110 threshold is slightly lower for the ILW discharges and the normalized growth rates are 111 smaller at the same  $R/L_{T_i}$ . For the experimental values of  $R/L_{T_i}$  (marked in Fig. 3), 112 we obtain  $\gamma_{ITG} = 0.16$  for ILW discharge 85407 and  $\gamma_{ITG} = 0.11$  for the matched CW 113 discharge in units of  $c_s/R$ . Similar results are obtained for the other pair of discharges. 114 In order to investigate the physics behind the difference in linear stability for the 115 matched pairs, a sensitivity study is performed with respect to the key dimensionless 116 parameters. The analysis include variations in plasma  $\beta$ , collisionality, magnetic shear, 117 Shafranov shift,  $R/L_{T_e}$ , ion to electron temperature ratio, safety factor, impurity content 118 and triangularity. The parameters are varied around the experimental values with up 119 to 20 %. The analysis is limited to one of the discharge pairs, but we have confirmed 120 that the conclusions are similar for the pairs under investigation. 121

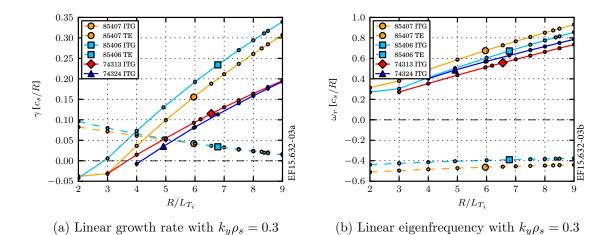


Figure 3: Linear  $R/L_{T_i}$  scans for the four discharges at  $k_y \rho_s = 0.3$ . Experimental  $R/L_{T_i}$  indicated.

First, in Figure 4, the growth rate spectrum is shown with plasma  $\beta$  as a parameter. The results show the well known linear stabilization of the ITG mode with plasma  $\beta$ . The experimental values are  $\beta = 0.78 \%$  for the ILW discharge and  $\beta = 1.2 \%$  for the C-wall case. The reason for the larger  $\beta$  value in the C-wall discharge can be traced to the difference in pedestal hight which is significantly lower in the ILW discharges.

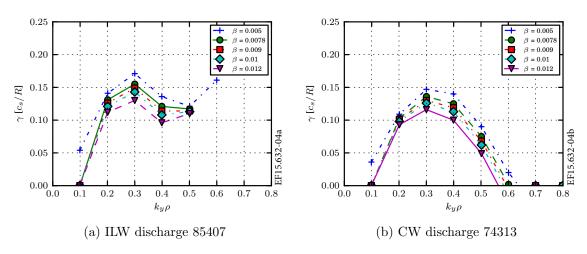


Figure 4: Scaling of eigenvalue spectra with  $\beta$ 

<sup>127</sup> The difference in plasma  $\beta$  between the matched discharges also has an effect on <sup>128</sup> the magnetic geometry through the Shafranov shift. Hence, the Shafranov shift is larger <sup>129</sup> for the C-wall case which enhances the stability of the ITG modes, as is shown in Figure <sup>130</sup> 5.

Next, the sensitivity with respect to magnetic shear is displayed. Magnetic shear is slightly destabilizing for ITG modes in the parameter regimes considered. As can

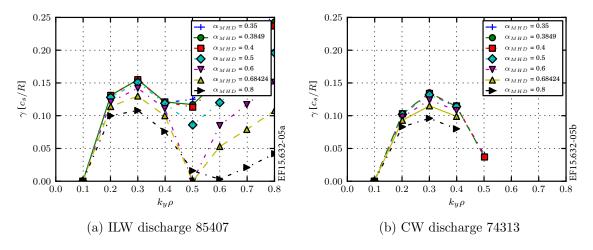


Figure 5: Scaling of eigenvalue spectra with  $\alpha_{MHD}$ 

be seen in Figure 6, the magnetic shear is larger for the ILW discharge, with  $\hat{s} = 0.66$ whereas  $\hat{s} = 0.56$  for the C-wall case.

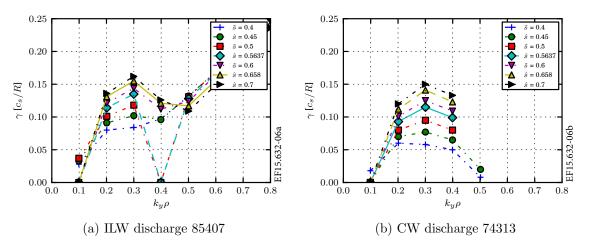


Figure 6: Scaling of eigenvalue spectra with  $\hat{s}$ 

In Figure 7 the destabilizing effect of the electron temperature gradient on the ITG stability is illustrated. The electron temperature gradient is larger for the ILW discharges  $(R/L_{T_e} = 8.3 \text{ versus } R/L_{T_e} = 6.2 \text{ in the C-wall case})$  which destabilizes both the ITG mode and the TE mode.

Figure 8 displays the corresponding growth rate spectra with collisionality given in Gaussian units with  $\nu_c = \pi \ln \Lambda e^4 n_e R/(2^{3/2}T_e^2)$  as a parameter. The collisionality is stabilizing for both discharges, with  $\nu_c = 0.003$  for the ILW case and  $\nu_c = 0.0019$ for CW. Since the collisionality is larger for the ILW discharges the relative effect of collisionality is stabilizing for ILW discharges. The reason for the larger collisionality in the ILW case is the lower temperatures in the ILW discharge.

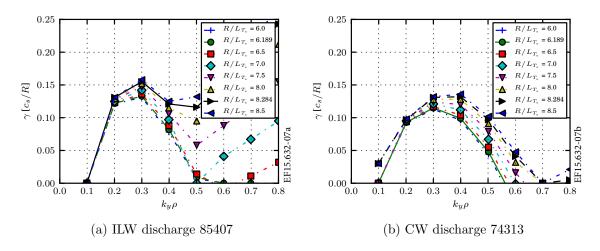


Figure 7: Scaling of eigenvalue spectra with  $R/L_{T_e}$ 

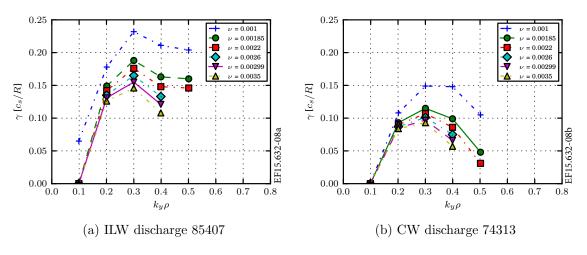


Figure 8: Scaling of eigenvalue spectra with  $\nu_c$ 

Finally, the effect of temperature ratio and impurity content on linear stability is investigated. The ion to electron temperature ratio is slightly larger for the ILW discharges  $(T_i/T_e = 1.0 \text{ versus } T_i/T_e = 0.91 \text{ in the CW case})$ . This is stabilizing the ITG mode but destabilizing the TE mode, as shown in Figure 9.

The impurity fraction and composition (C versus Be) is different in the matched pairs. It is well established that the impurity fraction is lower in the ILW discharges [3]. The impurities have a stabilizing influence on the ITG mode, mainly through main ion dilution. The result is a slightly more stable ITG mode in the C-wall case.

In summary, the ILW versus C-wall pairs considered are not perfectly matched with respect to dimensionless parameters. This leads to differences in linear stability of the main instabilities in the discharges. The reason for the mismatch in many parameters is related to the difference in pedestal height. This difference in the edge region translates

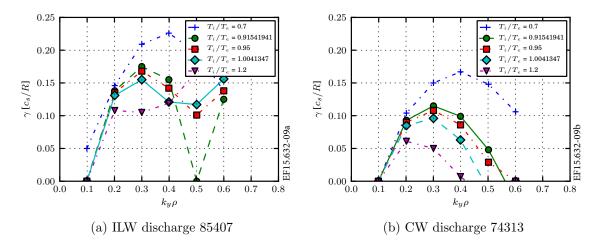


Figure 9: Scaling of eigenvalue spectra with  $T_i/T_e$ 

<sup>157</sup> into differences in the core of key parameters like  $\beta$ , Shafranov shift, and collisionality. <sup>158</sup> These differences are expected to disappear if the pedestal confinement is recovered, <sup>159</sup> e.g. through N seeding [19]. The difference in impurity content between the pairs leads <sup>160</sup> to a slightly more stable situation in the C-wall case which should remain even if the <sup>161</sup> pedestals are similar.

In Figure 10, the effect of the difference in dimensionless parameters on the linear 162 stability is summarized. The figure shows the relative change in the ITG growth 163 rate when the values of the parameters in one discharge is changed to that of the 164 corresponding paired discharge. As seen, the mismatch in  $\beta$ , Shafranov shift, magnetic 165 shear, and electron temperature gradient serve to destabilize the ILW discharges 166 relative to the CW discharges while the mismatch in collisionality and ion to electron 167 temperature ratio tend to stabilize the ILW discharges. The difference in the safety 168 factor and triangularity did not substantially change the linear stability properties. 169

#### 170 4. Nonlinear results

For the nonlinear GENE simulations, a simulation domain in the perpendicular plane 171 of  $[L_x, L_y] = [146, 126]$  were used, with a resolution of  $[n_x, n_y] = [96, 48]$ . In the parallel 172 direction 32 grid points were used, and in the parallel velocity direction 64 grid points, 173 and 16 magnetic moments. The simulations were typically run up to a simulation time 174 of  $t = 300 \,\mathrm{R/c_s}$  where R is the major radius and  $c_s = \sqrt{T_e/m_i}$ . The resolution and 175 simulation domain are checked through convergence tests. The two matched pairs of 176 ILW and CW discharges are simulated with input data taken at  $\rho = 0.5$ . The simulations 177 included effects of collisions, finite  $\beta$ , Miller equilibrium and impurity species, with an 178 impurity concentration of 0.4% of Be in the ILW discharges and 1.9% of C in the CW 179 discharges. In order to quantify the effects of rotation, its effect is included in one 180 simulation of each discharge. For some of the simulations, a higher  $R/L_{T_i}$  than the 181

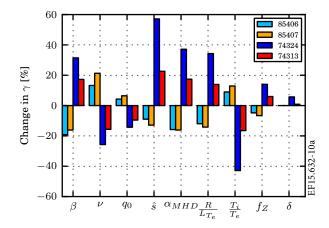


Figure 10: Growth rate change at  $k_y \rho_s = 0.3$ 

experimental was chosen because of the strong stabilizing effect of the ExB shear. As 182 can be seen in Figure 14b, this results in a reduction in the ion heat flux of around 20%. 183 For these simulation, both the effect from the toroidal shear and Coriolis and centrifugal 184 forces are included. Suprathermal pressure from fast ions, which has been reported to 185 lead to a significant reduction in the ion heat flux in gyrokinetic simulations of JET 186 discharges [20, 21, 22], is not included in the present simulations. While it was shown 187 in [20] that effects of fast ions were important at low radia ( $\rho = 0.3$ ) and low magnetic 188 shear, a weak effect was observed at larger radia and magnetic shear relevant to the 189 present case. 190

Due to the large uncertainty in the parameter  $R/L_{T_i}$ , the nonlinear simulations 191 of the ILW and CW discharges are performed as scans over  $R/L_{T_i}$ . A typical result 192 for the time series and flux spectra is shown in Figure 11 and 12 for the case with 193  $R/L_{T_i} = R/L_{T_e}$ , for the matched pairs 85407 (ILW), 74313 (CW) and 85406 (ILW), 194 74324 (CW). In order to investigate any differences in flux spectra between the matched 195 pairs, the mean  $k_{y}\rho_{s}$  for the ion heat flux was calculated along with a measure of the 196 width of the spectra. The width is taken as the wavenumbers responsible for 25% of 197 the flux over and under the indicated mean. The result is shown in Figure 13. As 198 seen, the differences in mean wavenumber and spectrum width between the ILW and 199 CW discharges are small. Figure 14 shows the scaling of ion and electron energy flux 200 with  $R/L_{T_i}$  in both normalized gyroBohm units and SI units. The electron temperature 201 gradient is here fixed at the experimental value. The error margin is obtained from 202 the time series, taking the statistical inefficiency of the data into account. An estimate 203 of the stiffness is obtained from the normalized fluxes in Figure 14. As observed, the 204 stiffness of the ILW discharges is larger than the matched ILW-discharges. In non 205 normalized units the heat flux for all the four discharges is comparable at the same 206  $R/L_{T_i}$ . The ion heat flux is larger than the electron heat flux as expected for ITG 207 dominated discharges. In Figure 14b, the ion heat flux at  $\rho = 0.5$  taken from the 208

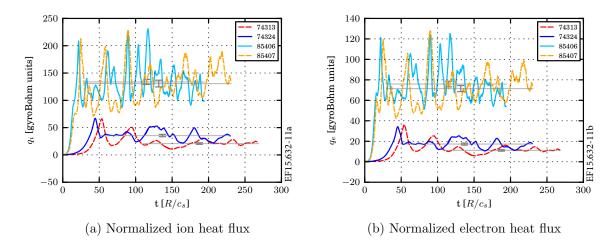


Figure 11: Time series data of the normalized ion and electron heat flux for the two pairs of CW and ILW discharges.

corresponding TRANSP runs is also shown. For the discharges at lower  $R/L_{T_i}$  the 209 experimental heat flux is comparable to the simulated flux while for the discharges at 210 higher  $R/L_{T_i}$ , the simulated ion heat flux is a factor ~ 3 higher. The discrepancy 211 between the experimental and simulated fluxes can be explained by the uncertainty in 212 the input parameters, in particular the uncertainty in the ion temperature gradient is 213 large for the ILW discharges. The results follow the linear trends in that the linearly more 214 unstable ILW discharges show significantly larger normalized fluxes. This is quantified in 215 Table 3 where the ion and electron heat fluxes and heat diffusivities are shown together 216 with the linear ITG growth rates for the four discharges. 217

The core energy confinement times in the volume within  $\rho = 0.5$  are calculated per species as

220

$$\tau_{core}^{j}(\rho < 0.5) = \frac{\frac{3}{2}k_B \int_0^{V'(\rho=0.5)} n_j(\rho) T_j(\rho) dV}{q_j(\rho=0.5)}$$

The results are shown in Figure 15. The electron energy confinement times are shorter 221 for the ILW discharges while the ion energy confinement times are similar. As noted, 222 the heat fluxes in SI units are similar at the same  $R/L_{T_i}$ , comparing the CW and 223 ILW discharges. The shorter electron energy confinement times are thus due to the 224 larger difference in  $T_e$  than  $T_i$  in the plasma within  $\rho < 0.5$  comparing the ILW and 225 CW discharges, as seen in Figure 2a and 2b. These conclusions are in line with the 226 experimental analysis of [4]; the difference can be attributed to the difference in NBI 227 heating power deposited to the electrons and ions in the ILW versus CW cases. The 228 fraction of total NBI power deposited to the electrons is larger for ILW discharges as 229 compared to the CW discharges. This is a result of the lower edge  $T_e$  in the ILW 230 discharges. 231

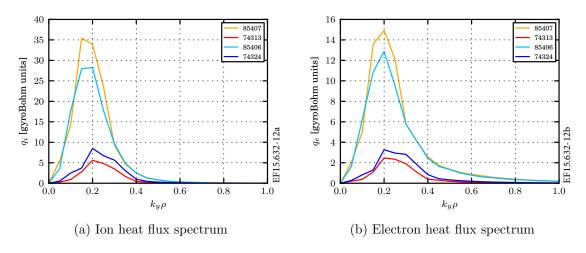


Figure 12: Time averaged heat flux spectra

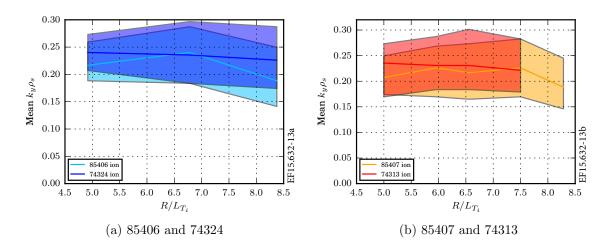


Figure 13: The mean wavenumber and width of the ion flux spectra for the two pairs of discharges, scaled with  $R/L_{T_i}$ .

Shot number	$R/L_{T_i}$	$R/L_{T_e}$	$q_i$	$q_e$	$\chi_i$	$\chi_e$	$\gamma_{ITG}$
74313	6.56	6.19	$27.7 \pm 1.7$	$14.6\pm0.8$	5.1	2.3	0.11
85407	5.96	8.28	$39.9\pm2.2$	$24.4 \pm 1.3$	6.8	2.9	0.16
74324	4.92	5.96	$10.8\pm1.8$	$5.51\pm0.89$	3.0	1.0	0.035
86406	6.78	8.37	$71.6\pm4.9$	$44.4 \pm 3.0$	9.9	4.8	0.23

Table 3: Linear and nonlinear results for the four discharges with experimental ion and electron temperature gradients. The heat fluxes and heat diffusivities are given in gyroBohm units,  $c_s n_e T_e \rho_s^2/R^2$  and  $c_s \rho_s^2/R$ , respectively. The linear data is for  $k_y \rho_s = 0.3$ in units of  $c_s/R$ .

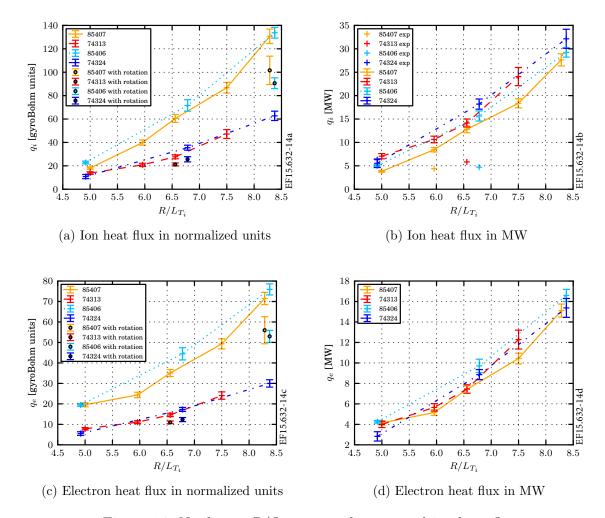


Figure 14: Nonlinear  $R/L_{T_i}$  scans, electron and ion heat flux

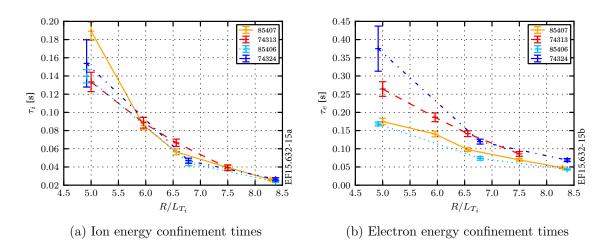


Figure 15: Ion and electron energy confinement times in the volume within  $\rho < 0.5$  for the four discharges seen in  $R/L_{T_i}$  scans.

#### 232 5. Conclusion

In the present paper, the linear stability and nonlinear fluxes of two pairs of matched 233 ILW and CW baseline ITG dominated H-mode discharges were studied at mid radius 234 using gyrokinetic simulations. The gyrokinetic simulations were performed using the 235 GENE code in a flux tube domain. The simulations included effects of collisions, finite 236  $\beta$  and impurities, Be for the ILW discharges and C for the CW discharges. The profile 237 data was taken from TRANSP runs with electron and ion temperature measurements. 238 A realistic Miller geometry description was used with parameters extracted from EFIT 239 reconstructions. The focus was on explaining the differences seen in core confinement in 240 baseline H-mode plasmas since the change of plasma facing components from a carbon 241 wall to a metal wall. Experimentally, this has resulted in a degradation of the pedestal 242 confinement with lower electron temperatures at the top of the edge barrier region. 243 The linear sensitivity scans showed that the relative change in key plasma parameters 244 between the ILW and CW discharges had a significant effect on the ITG/TE mode 245 stability. The relative change in plasma  $\beta$ , Shafranov shift,  $R/L_{T_e}$  and magnetic shear 246 served to destabilize the ILW discharges, while the relative change in collisionality 247 and ion-to-electron temperature ratio served to stabilize them. The total effect of 248 these parameter mismatches was that the ILW discharges were destabilized compared 249 to the CW discharges at all  $k_y \rho_s$ . The nonlinear results followed the linear ones in 250 that the ILW discharges show higher normalized heat fluxes at both comparable and 251 experimental  $R/L_{T_i}$ . The ion energy confinement times were similar, comparing the 252 CW and ILW discharges while the electron energy confinement times were shorter for 253 the ILW discharges which is in line with experimental analysis. These results indicate 254 that the core confinement in the ILW discharges was affected by changes in key plasma 255 parameters due to the degradation of the edge pedestal if compared to CW discharges. 256 Hence, we expect the core confinement in the ILW discharges to be improved if the edge 257 pedestals were recovered. 258

#### 259 Acknowledgements

The simulations were performed on resources provided by the Swedish National 260 Infrastructure for Computing (SNIC) at PDC Centre for High Performance Computing 261 (PDC-HPC), on the HELIOS supercomputer system at Computational Simulation 262 Centre of International Fusion Energy Research Centre (IFERC-CSC), Aomori, Japan, 263 under the Broader Approach collaboration between Euratom and Japan, implemented 264 by Fusion for Energy and JAEA, and on the supercomputer JUROPA at Jülich 265 Supercomputing Centre (JSC). This work was funded by a grant from The Swedish 266 Research Council (C0338001). This work has been carried out within the framework 267 of the EUROfusion Consortium and has received funding from the Euratom research 268 and training programme 2014-2018 under grant agreement No 633053. The views and 269 opinions expressed herein do not necessarily reflect those of the European Commission. 270

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